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A NOVEL PREPARATION OF ANHYDROUS UF 4

by

E. R. Russell and M. L. Hyder

Savannah River Laboratory E. I. du Pont de Nemours & Co. Aiken, South Carolina 29801

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E. R. Russell and M. L. Hyder Savannah River Laboratory E. I. du Pont de Nemours & Co. Aiken, South Carolina 29801

Uranium is commonly recovered from its ores or from nuclear fuel reprocessing in the hexavalent state as aqueous uranyl ion or a solid oxide compound. Conversion to  $UF_6$  for isotopic enrichment, or to the metal for fuel fabrication, involves a sequence of operations that may include denitration, precipitation, dehydration, high temperature hydrogen reduction, and hydrofluorination to yield anhydrous  $UF_4$  (1). This intermediate may be readily converted to metal or  $UF_6$ . We report here a method for the direct production of anhydrous  $UF_4$  by the electrolytic reduction of uranyl formate in the presence of fluoride ion.

Electrolytic reduction of uranium in aqueous solution to produce UF<sub>4</sub>-hydrate has been reported by others. Michal (2) dissolved UO<sub>3</sub> in H<sub>2</sub>SO<sub>4</sub>-HF solution, then precipitated UF<sub>4</sub>-hydrate by electrolytic reduction. Higgins, Neill, and McNeese (3) studied continuous electrolytic reduction of uranium from chloride solutions. Both sulfate and chloride solutions require special materials for handling because they corrode stainless steel.

Allen, Anderson, McGill, and Powell (4) have made an extensive study of the low-temperature electrolytic reduction and precipitation of UF4 from HF solutions. At temperatures below 35°C, the reduction of uranyl ion in the presence of excess HF causes precipitation of UF4.2.5 H2O. This hydrate could not be dehydrated thermally without oxidation (4). To prepare anhydrous UF4, it was first necessary to digest UF4.2.5 H2O in boiling water to produce UF4.0.75 H2O. UF4.0.75 H2O was then dehydrated to UF4 by heating in a nitrogen atmosphere (5).

Formic acid solutions are not corrosive to stainless steel, and uranium(IV) is stable in formic acid solution. Because formate-fluoride solutions are less corrosive than sulfate or chloride-fluoride solutions, we undertook a study of the use of formate-fluoride solutions for the electrolytic reduction of uranyl ion. We previously reported the reduction of uranyl ion by formic acid or formate under various conditions, including the thermal decomposition of solid uranyl formate to  $UO_2$  (6).

## RESULTS AND DISCUSSION

Uranyl nitrate was metathesized to uranyl formate by sorption of  ${\rm UO_2}^{2^+}$  on "Dowex" 50W\* cation exchange resin and elution with 1M formic acid and various concentrations of ammonium formate. (Formic acid is very inefficient in eluting uranium from "Dowex" 50W resin. Addition of ammonium formate to increase the formate

<sup>\*</sup> Trademark of Dow Chemical Co.

complexing action and supply sufficient  $NH_4^+$  to replace  $UO_2^{++}$  is necessary.) The solution was adjusted to 2M HF and transferred to a platinum dish for electrolysis. The uranium concentration was about 150 g/l. Electrolysis of  $\sim\!250$  ml at 6V was performed with the dish as cathode. A platinum rod suspended in the solution served as the anode. An emerald green ammonium uranous fluoride double salt deposited at the cathode. Electrolysis was continued until practically all of the uranyl color disappeared and the current decreased to less than one ampere.

The composition of the ammonium fluoride-uranous fluoride salt varied according to the ammonium concentration of the electrolysis mixture. The several compounds produced were identified by x-ray diffraction and are listed in Table I. The formation of the NH<sub>4</sub>F - UF<sub>4</sub> double salts is influenced by the NH<sub>4</sub> concentration and temperature (7). Low temperatures during precipitation produce mixtures of the double salts at all NH<sub>4</sub> concentrations.

All of the double salts are anhydrous and may be decomposed without oxidation by heating in the absence of oxygen. Anhydrous UF, was produced from each salt by slowly heating to 450°C in a nitrogen atmosphere to volatilize NH,F.

Precipitation of the double salt should provide additional decontamination of the uranium from fission products and other contaminating cations. To test this possibility, and to evaluate the UF4 product as feed for production of uranium metal, a uranium formate solution was contaminated with approximately 20,000 ppm

each of Fe, Al, Zr, Ru, and Cs. Uranium was electrolytically reduced and precipitated as  $NH_4U_2F_9$  from a 2M HF - 1M HCOOH - 0.5M HCOONH<sub>4</sub> solution. After  $NH_4F$  was removed by heating, a 30-gram uranium button (>85% yield) was produced by bomb reduction of the UF<sub>4</sub> with calcium. Emission spectroscopic analysis showed that the product contained  $\leq 0.01\%$  of each contaminant.

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TABLE I

Reduction and Precipitation of Uranium

(U = 150 grams/liter)

(2M HF)

Temp., °C	Product
90	NH4U2F9
90	NH <sub>4</sub> UF <sub>5</sub>
90	$(NH_4)_2UF_6$
50	Not identified
	90 90 90

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